



# **Risk-Informed Regulation at the U.S. NRC**

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**25<sup>th</sup> Anniversary of the  
Reliability Engineering Education Program  
The Center for Risk and Reliability  
University of Maryland  
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# NRC Oversight



Uranium Conversion



Uranium Enrichment



Power Reactors



Transportation



Storage



Waste Disposal



Medical/Industrial



New Reactors



# The Traditional Approach to Regulation (Before Risk Assessment)

- **Management of uncertainty (unquantified at the time) was always a concern.**
- **Defense-in-depth and safety margins became embedded in the regulations (*structuralist* approach)**
- **“*Defense-in-Depth* is an element of the NRC’s safety philosophy that employs successive compensatory measures to prevent accidents or mitigate damage if a malfunction, accident, or naturally caused event occurs at a nuclear facility.”** [Commission’s White Paper, February, 1999]
- **Questions that defense in depth addresses:**
  - **What if we are wrong?**
  - **Can we protect ourselves from the unknown unknowns?**



# Design Basis Accidents

- **A design basis accident is a postulated accident that a facility is designed and built to withstand without exceeding the offsite exposure guidelines of the NRC's siting regulation**
- **They are very unlikely events**
- **They protect against "unknown unknowns"**



# Technological Risk Assessment (Reactors)

- **Study the system as an integrated *socio-technical* system**
- **Probabilistic Risk Assessment (PRA) supports Risk Management by answering the questions:**
  - **What can go wrong? (thousands of accident sequences or scenarios)**
  - **How likely are these scenarios?**
  - **What are their consequences?**
  - **Which systems and components contribute the most to risk?**



# What Did We Learn from the Reactor Safety Study?

## Prior Beliefs:

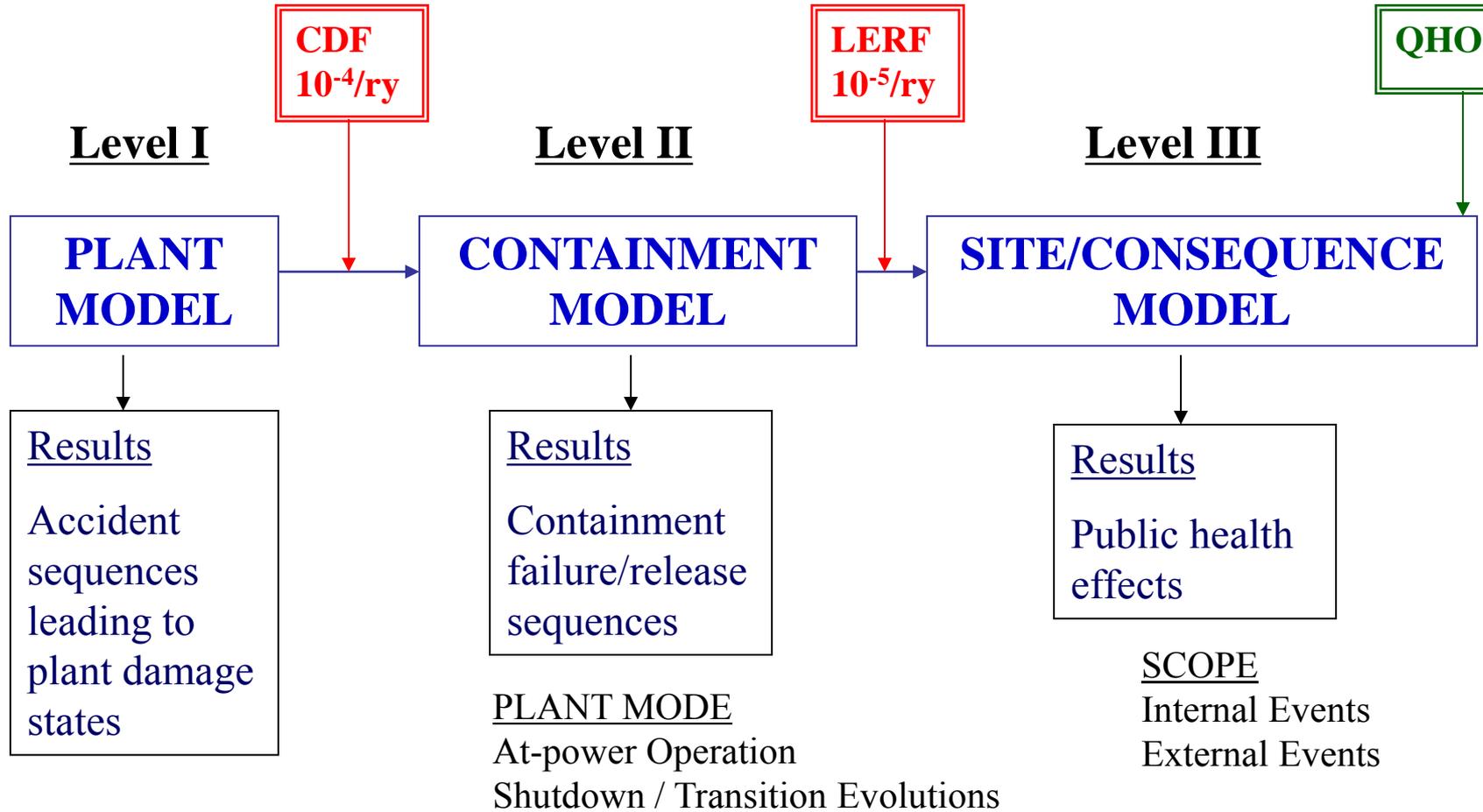
1. Protect against large loss-of-coolant accident (LOCA)
2. Core damage frequency (CDF) is low (about once every 100 million years,  $10^{-8}$  per reactor year)
3. Consequences of accidents would be disastrous

## Major Findings

1. Dominant contributors: Small LOCAs and Transients
2. CDF higher than earlier believed (best estimate:  $5 \times 10^{-5}$ , once every 20,000 years; upper bound:  $3 \times 10^{-4}$  per reactor year, once every 3,333 years)
3. Consequences significantly smaller
4. Support systems and operator actions very important



# PRA Model Overview and Subsidiary Objectives





# PRA Policy Statement (1995)

- **The use of PRA should be increased to the extent supported by the state of the art and data and in a manner that complements the defense-in-depth philosophy**
- **PRA should be used to reduce unnecessary conservatisms associated with current regulatory requirements**



# Risk-Informed Framework



## *Traditional "Deterministic" Approach*

- Unquantified probabilities
- Design-basis accidents
- Defense in depth and safety margins
  - Can impose unnecessary regulatory burden
- Incomplete

## *Risk- Informed Approach*

- Combination of traditional and risk-based approaches through a deliberative process

## *Risk-Based Approach*

- Quantified probabilities
- Thousands of accident sequences
  - Realistic
- Incomplete



# The Deliberation

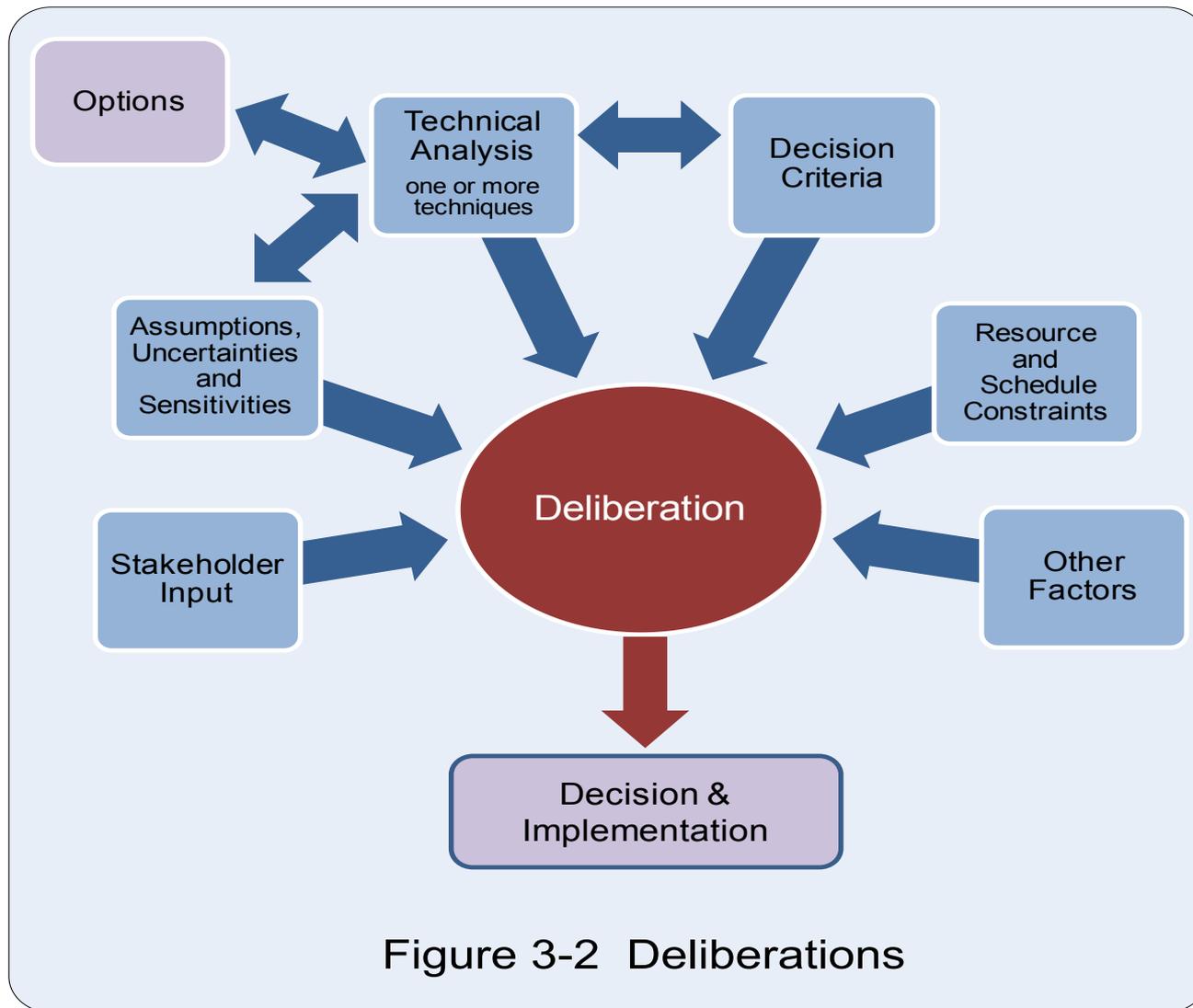


Figure 3-2 Deliberations



# Evolution of the NRC's Risk-Informed Regulatory System

- **1980s:** New or revised regulatory requirements based on PRA insights introduced
- **1990s:** Risk-informed changes to a plant's licensing basis allowed
- **2000:** Change to a risk-informed reactor oversight process made
- **2004:** Risk-informed alternative to comply with fire protection requirements introduced
- **2007:** Regulation requiring PRAs for licensing new reactors issued



# Risk-Informed Decision Making in Regulation

- **Improves Safety**
  - **New requirements (SBO, ATWS)**
  - **Design of new reactors**
  - **Focus on important systems and locations**
- **Makes regulatory system more rational**
  - **Reduction of unnecessary burden**
  - **Operating experience accounted for in regulations**
  - **Consistency in regulations**



# The Experience

- **Successes**
  - **Maintenance rule**
  - **Risk-informed inservice inspection**
  - **Reactor oversight process**
  
- **Challenges**
  - **Fire protection**
  - **Special treatment requirements**
  - **Risk-informing Emergency Core Cooling System rule**



# Summary

- **Uncertainties have always been of concern in safety**
- **Traditional methods manage uncertainties through design basis accidents and conservatism**
- **Risk assessment provides a global view of accident sequences, quantifies uncertainties, and is more realistic**
- **Risk-informed regulation combines the best features of both approaches**